

Electron irradiation-induced defects in Mo-diluted FeCrNi austenitic alloy during void swelling incubation

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Abstract. The microstructural features and the effect of Mo addition in FeCrNi austenitic alloy during incubation period were investigated using positron annihilation technique and micro-Vickers Hardness. The electron irradiation, which could induce vacancy defects in material, was performed at room temperature up to the dose of 1.7×10^{-4} and 5×10^{-4} dpa, respectively. The defect concentration was estimated about 10^{-4} - 10^{-7} though the standard trapping model. The added Mo atoms could trap vacancies to form Mo-vacancy complexes, which may restrain the migration and growth of vacancy defects during electron irradiation. In addition, the microstructural evolution during electron radiation resulted in hardening, while the added Mo might improve the hardening property of the alloy.

1. Introduction

Austenitic stainless steels are usually used as the structural material of nuclear reactors. Irradiation damage, such as void swelling and hardening, caused by prolonged energetic particle irradiation may affect the safety of nuclear reactors [1-2]. A transient stage or incubation period, which could determine the duration of the nuclear system, exists before steady void swelling occurs. Several theoretical and experimental analyses have been performed on the microstructural evolution during this period [3-6]. It is difficult to characterize the micro defects in this period by TEM for the reason that the defect size is usually lower than the resolution limit of TEM. Positron Annihilation Technique (PAT) is a suitable method to detect small-sized defects. It is well known that the defect structures of the incubation period relate with the irradiation temperature and dose. No microvoids would be found for the irradiation temperature above 473 K with the dose of 10^{-3} dpa in neutron irradiated Ti-modified 316 SS [7-8]. Meanwhile, the alloy elements may also affect the defect structures during the incubation period [1-2, 9-10]. As reported, undersize impurity atoms (P, Si) interacted with interstitial atoms, while oversized impurities (Ti, Nb) would interact with vacancies under irradiation in austenitic alloy [2]. These minor elements might induce the formation of stacking fault tetrahedral and precipitates, which could prevent the void growth [1, 9]. The added Mo could trap impurity atoms and reduce the number of dislocation loops under irradiation [10]. Additionally, Mo addition affect the formation of precipitates and crystal texture then enhance the mechanical properties and corrosion behaviour in different alloys [11-14]. As we known, the added Mo is the essential atom in FeCrNi austenitic stainless steel. This work was focus on the interaction of Mo addition with vacancy defects during incubation period.

In this study, FeCrNi and Mo-diluted model alloys were used to avoid the effect of other impurity elements. Electron irradiation was performed to generate vacancy defects. PATs, including Positron



annihilation lifetime spectroscopy (PALS), Doppler broadening spectroscopy (DBS) and coincidence Doppler broadening (CDB), were used to investigate the microstructural evolution under electron irradiation and the effect of Mo addition on microstructures. Irradiation hardening was characterized by micro-Vickers Hardness.

2. Experimental details

FeCrNi alloy (17wt% Cr, 12wt% Ni, the balance Fe) and Mo-diluted alloy (17wt% Cr, 12wt% Ni, 2.2wt% Mo, the balance Fe) were made from a series of high purity metals ($\geq 4N$) by arc melting process at General Research Institute for Nonferrous Metals. Specimens with the size of $10 \times 10 \times 0.3 \text{ mm}^3$ were electron-chemical polished to have a mirror like surface, and then annealed at 1323 K for 2 h in vacuum (about $1 \times 10^{-4} \text{ Pa}$).

Electron irradiation was performed by an electron linear accelerator of Research Reactor Institute, Kyoto University with an acceleration voltage of 8 MeV at less than 373 K. The damage rate was about $8 \times 10^{-8} \text{ dpa/s}$ and the irradiation dose was $1.7 \times 10^{-4} \text{ dpa}$ and $5 \times 10^{-4} \text{ dpa}$, respectively.

PATs were performed to characterize the microstructural evolution of irradiated alloy at room temperature. PALS were carried out by a conventional fast-slow spectrometer with a time resolution of 197 ps (FWHM). A ^{22}Na positron source with the activity of $\sim 10 \mu\text{Ci}$ was used. In order to reduce the statistical error, each spectrum accumulated about 2×10^6 coincidence events within 2 h. The spectra were decomposed into two components (τ_1 and τ_2) by LT9 after subtracting the source component and background. DBS were performed with a high purity germanium detector. The S parameter was defined as the ratio counts in the central energy region ($511 \pm 0.76 \text{ keV}$) to the total counts of the spectrum ($511 \pm 7.66 \text{ keV}$). CDB were performed to identify the chemical elements with two high purity germanium detectors placed 180° [15-16]. Each CDB spectrum was accumulated 1×10^7 counts to reduce the statistical error. Additionally, micro-Vickers Hardness was performed by a conventional apparatus with a load of 10 gF for 10 s in each measurement.

3. Results and discussion

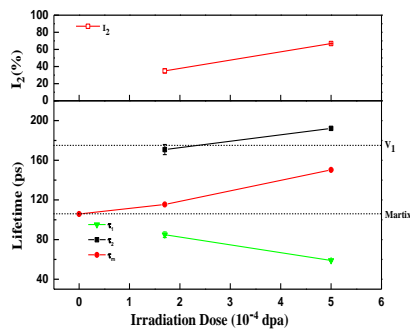


Figure 1. Dose dependence of the positron annihilation lifetime for electron irradiated FeCrNi model alloy.

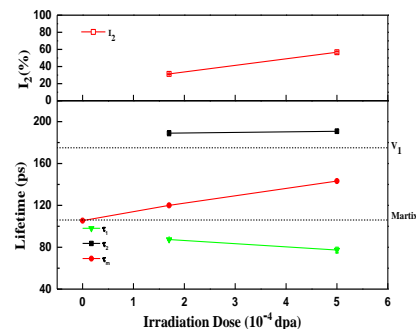


Figure 2. The positron annihilation lifetime for electron irradiated Mo-diluted alloy with different irradiation dose.

Figures 1 and 2 plot the dose dependence of the positron annihilation lifetime for electron irradiated FeCrNi and Mo-diluted alloy, respectively. τ_1 , τ_2 and τ_m denote the short, long and mean positron lifetimes, respectively. Wang et al. had reported that the positron lifetime of well annealed Fe15Cr30Ni model alloy was $110 \pm 3 \text{ ps}$ [17]. Meanwhile, the positron lifetime of matrix and mono-vacancy in AISI 316L was about 108 and 175 ps [18], respectively. Dashed line of Matrix and V_1 denote the calculated positron lifetimes of the matrix and the mono-vacancy in pure Ni, which was similar with the experimental result in AISI 316L. The mean lifetimes for both unirradiated alloys are about 105 ps, as shown in Figures 1 and 2. In Figure 1, the long lifetime at low irradiation dose is $171 \pm 5.1 \text{ ps}$, which is accord with the calculated mono-vacancy value (V_1). The result of higher irradiation dose is $189 \pm 5.1 \text{ ps}$,

which is larger than V_1 but smaller than V_2 . These results indicate that certain amount of small-size vacancy defects generated under electron irradiation. The increment of I_2 for both alloys indicates much more vacancy defects generated with elevated irradiation dose. However, I_2 in Mo-diluted alloy are slightly lower than that in FeCrNi alloy at the same irradiation condition. The added Mo might suppress the generation of vacancy defects during electron irradiation.

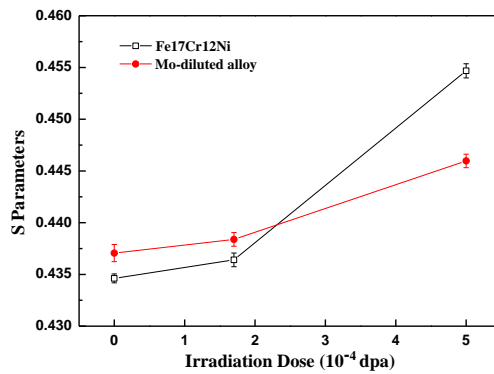


Figure 3. Dose dependence of the S parameters for FeCrNi and Mo-diluted alloys.

Figure 3. shows the S parameters of both alloys irradiated with different dose. The values of unirradiated alloys were attributed to the positron annihilated in matrix. The S parameters for both model alloys increased obviously with increasing irradiation dose, which means the increment of the defect concentration. According to the standard trapping model (STM) [19], the concentration of mono-vacancies can be estimated as:

$$C_V = \frac{\lambda_f(S - S_f)}{\mu_V(S_V - S)} \quad (1)$$

where λ_f is the annihilation rate in the defect free state; μ_V is the specific positron trapping rate into mono-vacancies. Because of the lattice structure and defects information is similar with pure Ni. Thus, the constant of Ni were chosen for our estimation ($\lambda_f = 9.1 \times 10^9 \text{ s}^{-1}$; $\mu_V = 2.2 \times 10^{15} \text{ s}^{-1}$ [20]); S_f and S_V are the S parameters characteristic of the positron annihilation from matrix and mono-vacancy-trapped state, respectively. $S_f = 0.435$ and 0.437 for FeCrNi and Mo-diluted alloy, respectively. As reported, S_V in pure Ni is equal to the saturated S value of cold rolled specimen (more than 10%)[21]. The saturated S value in plastic deformed Fe17Cr14.5Ni alloy has been identified about 0.455 ± 0.001 at room temperature. Thus, $S_V = 0.455$ was chosen in the calculation. The defect concentration was calculated about 10^{-7} for both alloys at lower irradiation dose, while 10^{-4} for FeCrNi alloy and 10^{-6} for Mo-diluted alloy at elevated irradiation dose.

Usually, point defects might be trapped by solute atoms because of the attribution of the solid solution effect [22]. Meanwhile, the oversized solute might reduce the migration of vacancies due to the high binding energy between the oversized atoms and vacancies [16]. In the present study, the added Mo, as oversized solute, existed in the alloy as solid solution state. The addition of Mo would trap vacancies and restrain the vacancy migration and growth. Therefore, swelling might be suppressed for the solid solution effect in Mo-diluted alloy.

CDB spectra could identify the chemical composition of the positron trapping site by measuring the momentum distribution of the electron annihilated with positron [15-16, 23-24]. As shown in Figure 4, the CDB ratio curves of irradiated specimens were normalized to the momentum distribution of FeCrNi alloy. One peak within the momentum region of $8 \times 10^{-3} m_{0c} < |P_L| < 18 \times 10^{-3} m_{0c}$ is observed in Figure 4 (b). The solute Mo atoms might trap vacancies to form Mo-vacancy complexes, which might change the chemical composition around vacancy sites compared to FeCrNi. Part of incident positrons, trapped by vacancies, annihilated with the characteristic core electron of Mo atoms. The effect of Mo-vacancy complexes induced the generation of the peak at high-momentum region.

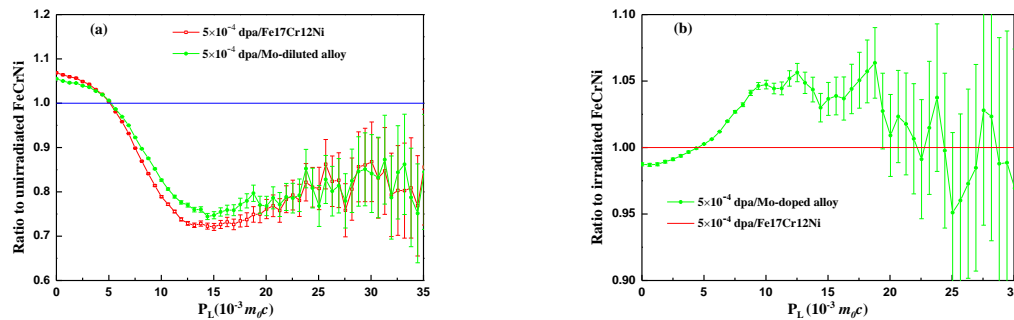


Figure 4. Ratio curves of CDB spectra in as-irradiated model alloy. (a) Normalized to the momentum distribution of as-annealed FeCrNi. (b) Comparison of two alloys at high irradiation dose. m_0 is the electron rest mass and c is the speed of light.

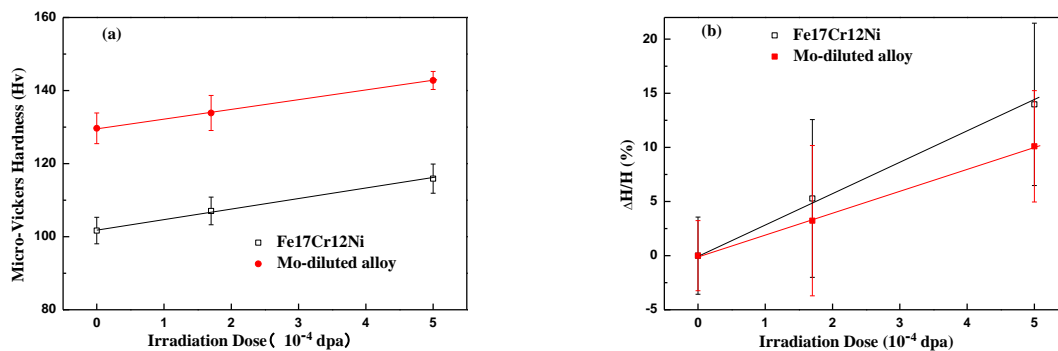


Figure 5. Dose dependence of micro-Vickers Hardness plot for different model alloys. (a) the value of micro-hardness, (b) $\Delta H/H$ parameters for both alloys with different radiation dose.

The hardness usually related with the microstructural features. The micro defects, such as dislocation loop, vacancy cluster or micro void and precipitates could induce hardening in materials [23, 25-26]. The micro-Vickers hardness value indicates that a significant difference exists between the two alloys, as shown in Figure 5. The increment of hardness for both alloys attribute to the vacancy defects induced by electron irradiation. $\Delta H/H = (H_{\text{irra}} - H_{\text{unirra}}) / H_{\text{unirra}}$, increase linearly for both alloys, as shown in Figure 5 (b). The slope of FeCrNi is larger than that of the Mo-diluted alloy. Irradiation hardening could be suppressed because of the Mo addition, which is consistent with the defect concentration. Additionally, the hardness value of Mo-diluted alloy was larger than that of the FeCrNi alloy at the same irradiation condition, as shown in Figure 5 (a). The solution strength induced by the added Mo atoms might be the main reason for this phenomenon [27-28].

4. Conclusion

PATs were performed to characterize the microstructural evolution in electron irradiated FeCrNi and Mo-diluted alloy as a function of dose dependence. The hardening process related with defect concentration was characterized by micro-Vickers Hardness. Certain amount of small-sized vacancies generated in the material after electron irradiation. The defect concentration was estimated about 10^{-4} for FeCrNi alloy and 10^{-6} for Mo-diluted alloy at elevated irradiation dose, while 10^{-7} for both alloys at lower dose. The diluted Mo could trap vacancy defects to form Mo-vacancy complexes and restrain the migration and growth of vacancy defects during electron irradiation. Hardening process was suppressed because of the Mo addition. Swelling would be suppressed in Mo-diluted FeCrNi austenitic alloy under irradiation.

Acknowledgements

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